

Program of Vietnam/Japan Research/HRD Forum on Nuclear Technology-I
Place of Forum:702 Meeting room, Ta Quang Buu Library, Hanoi University of Science and Technology

Time	Title	Name	MC/Responsible
December 23(Monday), 2013			
8:00~8:30	Registration		
8:30~9:00	Opening Ceremony		
8:30~8:40	Opening Remark	Masaki Saito	V: Tran Ngoc Toan (Vinatom)
8:40~8:50	Opening Remark	Tran Chi Thanh	J:Murata(Hitachi)
8:50~8:55	Opening Remark	Le Dinh Tien	
8:55~9:00	Photograph	-	
9:00~11:20	First session (Review paper)		
9:00~9:35	Nuclear Safety Research in Japan History and perspectives -	Jun Sugimoto	V: Le Van Hong (Vinatom)
9:35~10:10	Research on Severe Accidents at Nuclear Power Safety Division (NPS), Royal Institute of Technology (KTH), Sweden	Tran Chi Thanh	J:Kikura(Tokyo-Tech)
10:10~10:45	Thermal Hydraulic Research and Design in BWR	Kumiaki Moriya	
10:45~11:20	Thermal Hydraulics Safety Research for NPPs in Vietnam	Le Dai Dien	
11:20~12:30	Working Lunch hosted by VINATOM		
12:30~14:00	Second session (Poster session)		
	Development of focusing sensors for Air-coupled Ultrasonic Flowmeter	Keisuke TSUKADA ,	J:Saito,Ogawa,Moriya V: Tran Chi Thanh, Ha Manh Thu, Tran Kim Tuan
	Numerical and Experimental Investigation of Chaotic Flow in a Cubic Joule-heated Glass Melter	Thanh Tung DUONG	J:Saito,Ogawa,Akimoto V:Tran Chi Thanh, Ha Manh Thu, Nguyen Minh Tuan
	High-temperature chemical degradation of Basalt aggregates	Kentaro Ishizu	J*Sugimoto,Kikura,Yoshimoto V:Nguyen Phu Khanh, Le Dai

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			Dien, Nguyen Minh Tuan
	Experimental Study of Stress Intensity Factor of the Intergranular Stress Corrosion Cracking of Sensitized SUS304 Utilizing Fretting Corrosion	Masataka Hiruta	J:Sugimoto,Kikura,Tatsumi V: Le Van Hong, Nguyen Phu Khanh, Le Dai Dien
	Natural hazard assessment of Ninh Thuan nuclear power plant based on the IAEA safety standards	Vu Manh Kha	J:Akimoto,Moriya,Arai V: Le Van Hong, Hoang Minh Giang, Nguyen Viet Hung
	Study of annular two-phase flow on nuclear fuel-rod bundle by using high-speed camera technique	Son Hong Pham	J:Akimoto,Moriya,Kikura V: Hoang Minh Giang, Nguyen Phu Khanh, Ha Manh Thu
	INVESTIGATION OF TEMPERETURE STRATIFICATION IN COLD LEG AND DOWNCOMER IN LSTF/ROSA EXPERIMENT	Trần Văn Trung	J:Arai,Tatsui,Sugimoto J: Tran Chi Thanh, Hoang Minh Giang, Tran Kim Tuan
	Calculating amount of hydrogen gas will be generated in a boiling water reactor when the SBO (Station Black Out) accident happens by using MELCOR.	Doan Manh Long	J:Arai,Tatsumi, Saito, V: Le Dai Dien, Nguyen Viet Hung, Nguyen Minh Tuan
14:00~17:45	Third session (CFD)		
14:00~14:35	CFD Application in BWR Reactor System Design	Yuuichiro Yoshimoto	V: Hoang Minh Giang (Vinatom) J:Arai(Toshiba)
14:35~15:10	Analysis of void behavior in Transient Boiling Experiments simulating BWR RIA conditions by using the code TRAC-BF1	Nguyen Minh Tuan	
15:10~15:45	Two-Phase flow analysis code development for light water reactor	Hajime Akimoto	
15:45~16:00	Tea Break		
16:00~16:35	SAFETY ANALYSIS FOR WORKING CORE OF THE DALAT NUCLEAR RESEARCH REACTOR USING LOW ENRICHED URANIUM FUEL	Nguyen Kien Cuong	V: Nguyen Phu Khanh (HUST)

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16:35～17:10	Computational Fluid Dynamics Models for Boiling Flows in Nuclear Reactor Fuel Assemblies	Nguyen Van Thai	J:Sugimoto(Kyoto)
17:10～17:45	STUDY ON PWR SUBCHANNEL VOID DISTRIBUTION BENCHMARK S1-PSBT	Hoang Minh Giang	
18:00～20:00	Reception hosted by VINATOM		
December 24(Tuesday), 2013			
	Forth session (Severe accident)		
8:30～9:05	Melting attack of solid plates by a high-temperature liquid jet	Masaki Saito	V: Nguyen Viet Hung (HUST)
9:05～9:40	The Effective Convectivity Model (ECM) and its Application to Analysis of Severe Accidents	Tran Chi Thanh	J:Ogawa(Nagaoka-Tech)
9:40～10 : 15	Simulation of FOREVER experiment with the ANSYS software for nuclear safety in severe accident scenario	Le Anh Tuan	
10:15～10:35	Tea Break		
10:35～15:40	Fifth session (Experiment, Education, etc)		
10:35～11:10	Contributions to severe accident chemistry :Education program on fundamental research	Toru Ogawa	V: Tran Chi Thanh (Vinatom) J:Akimoto(JAEA)
11:10～11:45	LWR Thermal-Hydraulic Research in Japanese Nuclear Industry - Focusing on Experimental Research -	Kenji Arai	
11:45～12:20	Advanced Flow Measurements for Thermal-Hydraulic Studies on LWRs	Hiroshige Kikura	
12:20～13:20	Lunch Break hosted by Japan		
13:20～13:55	Establishment of Evaluation Method for Heterogeneous External Boron Dilution	Eisaku Tatsumi	V: Le Dai Dien (Vinatom)
13:55～14:30	Design and construction of a PWR testing facility (Part I.)	Trinh Huu Toan	J:Moriya(Hitachi)
14:30～15:05	Development of Standardized and Consolidated Reference Experimental Database (SCRED) for Code Assessment and Uncertainty Evaluation And application in LOFT facility	Nguyen Hoang Anh	
15:05～16:30	Closing session		

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15:05～15:30	Summarized by Vietnam	Tran Chi Thanh	V: Tran Ngoc Toan (Vinatom)
15:30～15:55	Summarized by Japan	Masaki Saito	J:Yamamoto(Toshiba)
15:55～16:15	Awards	Saito/Thanh	
16:15～16:20	Conclusive remark	TBD	
17:00～19:00	Reception hosted by Japan		V: Hoang Sy Than (Vinatom) J:Saito(MHI)